

CATAWBA NUCLEAR STATION
UNIT 2
EVALUATION OF 2EOC13 ISI
FLAW IN SG2C TO HOT LEG WELD
CNC-2201.01-00-0006

CERTIFICATION OF ENGINEERING CALCULATION

Station And Unit Number Catawba Nuclear Station, Unit 2

Title Of Calculation Evaluation of 2EOC13 ISI Flaw in SG2C to Hot Leg Weld

Calculation Number CNC-2201.01-00-0006

Total Original Pages 1 Through 8

Total Supporting Documentation Attachments 1 (A) Total Microfiche Attachments -

Total Volumes 1 Active Calculation/Analysis Yes ☐ No ☒

Microfiche Attachment List ☐ Yes ☒ No If Active, is this a Type 1 Calculation/Analysis Yes ☐ No ☐

(See Form 101.4)

These engineering Calculations cover QA Condition 1 Items. In accordance with established procedures, the quality has been assured and I certify that the above Calculation has been Originated, Checked, or Approved as noted below:

Originated By Brian Callaway Date 10-13-04

Checked By James F. Willis Date 10-13-04

Verification Method: Method 1 ☒ Method 2 ☐ Method 3 ☐ Other ☐

Approved By Daniel R. Ward Date 10-13-04

Issued To Document Management Dhw Date 10-13-04

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Pages _____ Through _____

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Pages _____ Through _____

Originated By _____ Date _____

Checked By _____ Date _____

Verification Method: Method 1 ☐ Method 2 ☐ Method 3 ☐ Other ☐

Statement of Problem / Purpose

This calculation has been performed to address an indication discovered on the 2C Steam Generator to Hot Leg weld during 2EOC13. This particular weld is located between the cast low alloy steel primary channel head of the steam generator and the cast austenitic stainless steel elbow. Typically, the volumetric examination under Section XI of the ASME Code would be performed using an ultrasonic examination. However, the cast austenitic stainless steel material of the reactor coolant loop piping precludes the use of the ultrasonic examination methodology. The ultrasonic examination was replaced with a radiographic examination.

QA Condition

This calculation is QA Condition 1 because it serves as the basis for continued qualification of a Duke Class A, ASME Class 1 component.

Analysis Methodology

This calculation verifies that the structural integrity of the steam generator 2C nozzle to hot leg weld is maintained under all operating and design loading conditions, even with the identified flaw. The verification has been performed by conservatively bounding the flaw characteristics and comparing this bounding flaw against allowable flaw depths provided by a Westinghouse flaw evaluation handbook [7] specific to this reactor coolant loop location.

Other Evaluation Criteria

No other evaluation criteria have been used.

Applicable Licensing References

- L. 1 Catawba UFSAR, September 27, 2003 issue date, Sections 3.9 and 5.2
- L. 2 Catawba Selected Licensee Commitments 16.5.5, Reactor Coolant System, Structural Integrity, Rev 0

Assumptions/Engineering Judgments

As noted in body of calculation

References

- 1 ASME Code, Section III, Subsections NB and NC, 1974 Edition with Summer 1974 Addenda.
- 2 ASME Code, Section XI, 1989 Edition, No addenda
- 3 Westinghouse sketch EDSK 380335B, from CNM-2201.01-0217
- 4 Westinghouse sketch EDSK 380329B, from CNM-2201.01-0217
- 5 CNM-1201.01-0076, Rev 0, ESCO drawing AA-43234, , Weld Bevel Detail, 31.00 I. D. Elbows, Nozzle Ends
- 6 CNM-2201.01-0008, Rev B, ESCO drawing AD-22321, Machining – 31.00 I.D. x 29.00 I.D. Long Radius 50° Reducing Weld Elbow
- 7 WCAP-15658-P, September, 2004, Flaw Evaluation Handbook for Catawba Unit 2 Steam Generator Primary Nozzle Weld Regions.
- 8 CN-2553-1.0, Rev 22, NC Flow Diagram.
- 9 CN-2NC-0013, Rev 13, Weld Isometric.
- 10 PIP C-04-5421

Calculation/Evaluation

This calculation was performed by determining the bounding characteristics of the indication and comparing parameters associated with this indication to values in the Westinghouse supplied flaw analysis handbook [7].

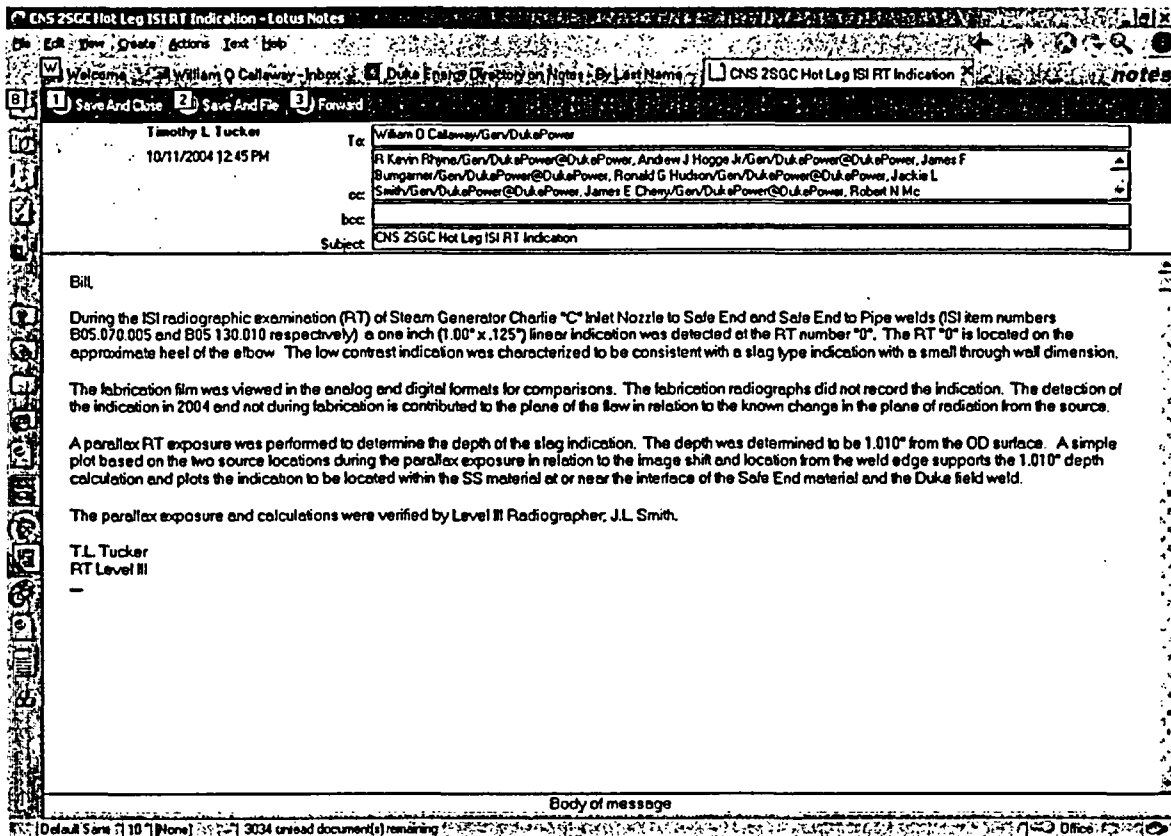
Nozzle Configuration

The nozzle connection consists of a low alloy steel casting that forms the channel head of the steam generator. This casting has been buttered with a low carbon, stainless steel weld metal. The piping to buttering field weld is made after post weld heat treatment of the steam generator channel head. The Duke weld number is 2NC-13-2 and the weld process control from the original fabrication is included as Attachment A. This weld is a full penetration, compound V groove weld made from the outside of the pipe. The GTAW (TIG) process was used for the first inch, followed by a "courtesy" radiograph (RT). Subsequent welding was performed using the SMAW (stick) process to finish out the weld. After completion of welding, a final RT was performed and accepted on the weld. In addition, liquid penetrant tests (PT's) were performed on the interior and exterior surfaces of the weld.

Flaw Geometry

The flaw is located at the bottom of the pipe in the C hot leg. It is approximately at bottom dead center of the pipe. Based on the radiographic data, the flaw is 1" long and oriented in the circumferential direction. Since the examination was performed using RT, a limited amount of information was available to characterize the flaw. The location of the flaw relative to the OD surface was established using parallax radiographic shots. These shots support a minimal flaw depth. However, because of the uncertainty in flaw depth, a bounding case has been reviewed herein.

Radiography Evaluation



In addition to the radiography shots made to characterize the flaw, both the original construction film and the 2EOC13 film (non-parallax shots) were digitized. The original construction film was digitized to determine if the indication could be seen from initial fabrication welding. Digitization of the film can greatly enhance the visible interpretation of the film in some cases. Next, the 2EOC13 film was digitized and reviewed to determine if the linear indication was actually separated into multiple flaws. In both of these cases, there was no conclusive evidence from the digitization process that changed the film interpretation or flaw characteristics.

Based on the conclusions provided from the radiography review above, a best estimate characterization of the flaw has been provided. The flaw is located 1.01" from the outside surface of the piping in the stainless steel weld material. The flaw is oriented circumferentially with a length of 1.0". The flaw is most likely the result of a slag inclusion during fabrication. It has very little contrast that indicates a limited depth. It is located at the interface between the stainless steel buttering and the Duke stainless steel field weld.

The flaw location from the OD surface of 1.01" was considered from three positions relative to the flaw depth (top, center & bottom). The three positions were considered for two different aspect ratios. All six cases were found to be acceptable. Based on the method used to determine location, the center position is the most appropriate and is used in the documented flaw calculation below.

For the initial evaluation, the flaw depth will be assumed as 1/2 of the length and evaluated as an embedded flaw. From the flaw handbook, several parameters are necessary to determine the acceptability of the indication. These are provided below. The appropriate figure for the purposes of evaluation from reference [7] for a circumferential embedded flaw in the stainless steel material is Figure A-3.7.

a = half flaw depth (in)
= 0.25 in.
l = length of flaw (in)
= 1.0 in.
t = wall thickness (in)
= 3.25 in.

Note: The wall thickness is based on profiling of weld using UT probe. Value of 3.25" is conservative and represents the lowest reading throughout the weld region of interest. See page 8 for the UT profile readings at the location of interest.

δ = distance to flaw centerline
= 1.01 in.
 δ / t = $1.01 / 3.25$
= 0.311
 a / t = $0.25 / 3.25$
= 0.077

The δ / t and a / t parameters may be plotted on Figure A-3.7 to determine the acceptability of the flaw. This point (A) is shown on the attached sketch.

In addition to the above evaluation, the flaw depth was increased to 1" yielding an aspect ratio of 1:1. In this case, the parameters change as noted below:

a = half flaw depth (in)
= 0.50 in.
l = length of flaw (in)
= 1.0 in.
t = wall thickness (in)
= 3.25 in.
 δ = distance to flaw centerline

$$\begin{aligned}
 &= 1.01 \text{ in.} \\
 \delta / t &= 1.01 / 3.25 \\
 &= 0.311 \\
 a / t &= 0.50 / 3.25 \\
 &= 0.154
 \end{aligned}$$

Again, the δ / t and a / t parameters have been plotted on Figure A-3.7 as point (B) to determine the acceptability of the flaw.

Results

In both cases evaluated above, it is clearly evident that the flaw is within the bounds of the acceptability provided by Figure A-3.7 of reference [7]. As a result, the piping containing this flaw is acceptable for continued service for the design life of the plant. The figure in reference [7] indicates 10, 20 and 30 year acceptance lines. These lines are related to the design number of occurrences of transients used in the fatigue crack growth calculation. As such, this indication is acceptable for life of plant provided a prorated value ($30 / 40 = 75\%$) of the design number of occurrences are not exceeded between now and end of plant life. This limit on fatigue cycle counts will be tracked under our fatigue management program.

Conclusion

The flaw discovered during 2EOC13 is acceptable without repair for the life of plant. Acceptance by the performance of analytical evaluation as allowed by ASME XI, IWB-3132.4 has been validated. Additional examinations have been performed during 2EOC13 to satisfy IWB-2430. Successive examinations for the SG2C hot leg weld number 2NC-13-2 will be necessary in the subsequent three ISI periods as required by IWB-2420.

Page 7

Proprietary



Supplemental Report

Report No. _____

Page: _____ of _____

Summary No.: B05.070.007

Examiner: [Signature]

Examiner: [Signature]

Other: _____

Level: II

Level: II

Level: _____

Reviewer: _____

Site Review: _____

ANII Review: _____

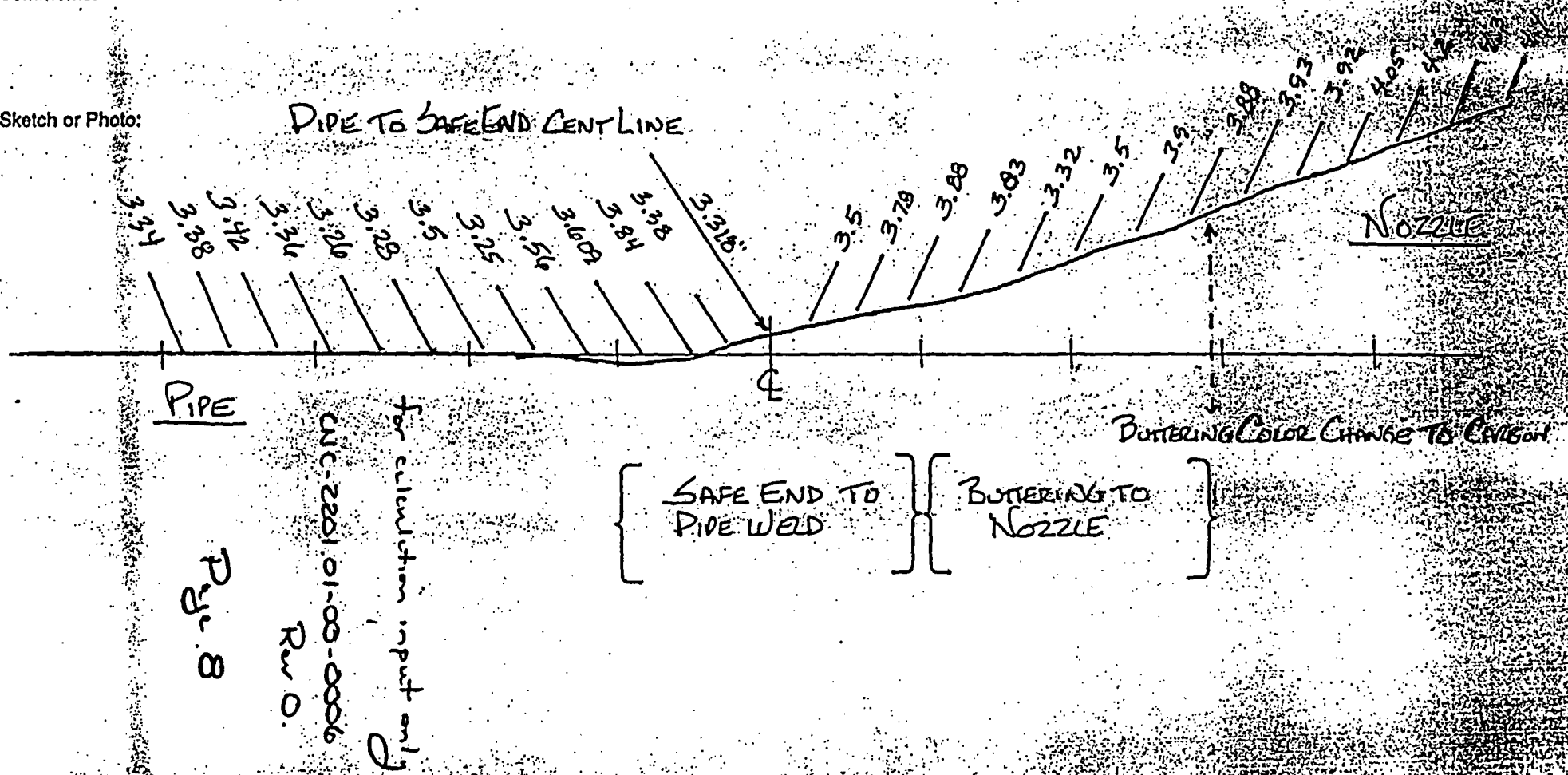
Date: _____

Date: _____

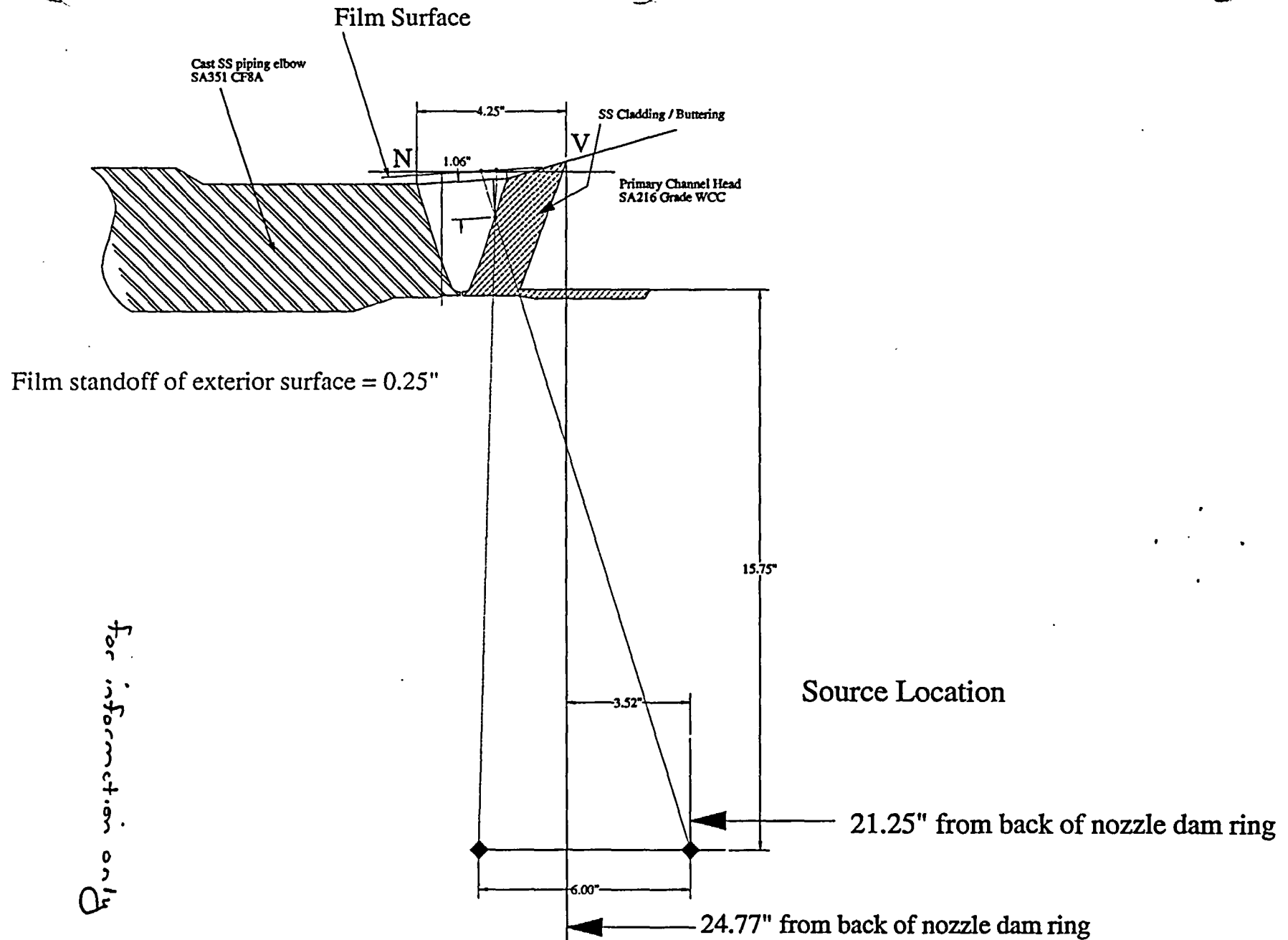
Date: _____

Comments:

Sketch or Photo:



CNS Steam Generator 2C Hot Nozzle Indication



For information only

Reference Drawings:

- 1) CNM-2201.01-0008
- 2) CNM-1201.01-0076

Page A-3
A-4

Proprietary

25585
5/23/80

RELATED MATERIAL ATTACHED

FORM 12-4A

REVISION 15

ORIGINATOR	DATE	DUKE POWER COMPANY CONSTRUCTION DEPARTMENT PROJECT CATAWBA
MECH. CHECK	8/27/80	WELD PROCESS CONTROL SHEET
WELD CHECK	8/27/80	
O & REVIEW	8/28/80	
		PRE-APPROVED DYES <input checked="" type="checkbox"/> YES <input type="checkbox"/> NO
		INFO IDENT
		INFO REV. NO.

Weld No. <u>2 NC 13 - 2</u> Class <u>A</u>	F.W.D.S. <u>L-206 AND 1-211 ROOT</u>
Drawing No. <u>CN-2NC-13</u>	<u>L-206 AND 1-304 FILL</u>
F-9B Required Yes <input checked="" type="checkbox"/> No <input type="checkbox"/> Type <u>BUTT</u>	Preheat <u>60°F MIN</u> PWHT Data Sheet <u>NA</u>
Pipe Size <u>31"</u> Joint Thickness <u>3.22"</u>	Repair NDE <u>NDE-30J</u>
Material <u>SS(P-8) 304</u>	Final NDE <u>NDE-30J</u> FILL <u>NDE-10</u>
Remarks <u>FINAL "PT" TO INCLUDE I.D. AND O.D.</u>	Removal <u>WELDING ON I.D. IS PERMISSIBLE</u>

ANI REVIEW				REPAIR				ANI INSPECTION			
NO.	DATE	INITIAL	DATE	NO.	DATE	INITIAL	DATE	NO.	DATE	INITIAL	DATE
X 1	Repair NDE Proc. NDE-30J Rev. 7	KW	9-2-80								
X 2	Cleanliness	KW	9-2-80								
X 3	Fit-Up	KW	10-2-80								
X 4	Purge	KW	10-2-80								
X 5	Preheat	KW	10-2-80								
X 6	Visual Inspection	KW	10-2-80								
X 7	Inspection Criteria CP 106	KW	10-2-80								
X 8	NDE Inspections	KW	10-2-80								
	a PT Proc. NDE-30J Rev. 9	KW	11-15-81								
	b WT Proc. N/A Rev. 1										
	c UT Proc. N/A Rev. 1										
	d RT Proc. NDE-30J Rev. 3 O.C.	KW	11-21-81								
	e PWHT Proc. Rev. 1	KW	11-21-81								

Welding Serial Number	Yes <input checked="" type="checkbox"/> No <input type="checkbox"/>	Form M-4L Used <input checked="" type="checkbox"/> Yes <input type="checkbox"/> No
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Repair No.	WELDER INITIAL	STEEL NUMBER	FILLER METAL	WELDER TYPE	PLANT/LOT NUMBER	DATE	INSPECTOR	DATE
0	GMR	N/A	5/32	AE308EB	E41031308	8-20-80	KW	10-2-80
0	WLR	MSD	N/A	N/A	N/A	8-20-80	KW	10-2-80
0	MRW	414	N/A	N/A	N/A	8-20-80	KW	1-8-81
0	MRW	414	N/A	ER308	04184	8-20-80	KW	1-8-81

RECORDS OF SURFACE DEFECTS BY NIT/PT		TRACEABILITY RECORDS	
A. Type of Defect		(A) Piping Materials (K) Heat Numbers	
B. Defects removed, surface re-temmed by 2T/PT		Heat Numbers	
C. MT machine serial no.		NDE Piece Mark	
D. Weld acceptable without weld repair		Backing eng heat no. Value	
		Serial Number	
		DATE 3/21/81	

Attachment A pg 1

CNC-2201.01-00-0006

rev 0

(A.)
Nitley 2C-1 ~~5-116~~ 10-2-80
 ~~183~~ GMR
18842 ✓
 A.M.
 P. 17:43

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11/15/79

FORM F-98

REVISION 7

INITIAL DATE	
ORIGINATOR	DEK 11/15/80
MECH. CHECK	RLR 8/27/80
WELD CHECK	RLR 8/27/80
O.A. REVIEW	RLR 8-28-80

DUKE POWER COMPANY
CONSTRUCTION DEPARTMENT
PROJECT CATAWBA

Sheet 1 of 1

PRE-APPROVED YES ☒ NO
INFO. IDENT.
INFO. REV. NO.

DETAILED PROCESS CONTROL SHEET

SYSTEM NC DRAWING/SKETCH NO. CN-2NC-13
WELD NUMBER 2NC 13-2 CONTROL SHEET REVISION NO. 0

ANI REVIEW
DATE 9-2-81
INSPECTION POINT YES ☒ NO

OPERATION NUMBER	OPERATION DESCRIPTION	INSPECTOR CRAFTSMAN	DATE	ANI HELD PT	ANI INITIATE	DATE
0	POLISH SAME END USING "FLAP" TYPE WHEEL. (CRAFT)	GNW	9-10-80			
0A	RINSE WITH DEMINERALIZED WATER. (CRAFT)	GNW	9-10-80			
1	PT OF WELD FREE TO BE DOCUMENTED AT THIS STEP. (QC)	END	9-11-80			
5A	AFTER APPROXIMATELY ONE INCH OF DEPOSITED WELD METAL. RT. PROD (D) RTES/REV <input type="checkbox"/> ACCEPT (QC) 11/11/80 10-21-80 DESIGNED BY RLR 8/28/80 (QA)	11/11	10-21-80			
3	allow lead thickness of .055" to .105" when using 1200#. Top of lead & ring may not be flush.	RLR	11-2-80			
5B	FOR COURTESY RT <input type="checkbox"/> ACCEPT (QC) 11/11/80 10-30-80 DESIGNED BY RLR 8/28/80 (QA) 11/15/80	11/11	10-30-80			
		RLR	3/21/81			

DEC 17 1980
RLR
9-17-80
OK
2/17/80

RLR

Attach A, pg 3

CNC-2201.01-00-0006

REV 0

M-4A CONTINUATION SHEET

ORDER NO	INITIAL	SERIAL NUMBER	WELL SIZE	FILTER TYPE	HEAT/LOT NUMBER	FIELD WELD DATA SHEET USED/REV	DATE	INSPECTOR	DATE
0	WJ	M80	1/16	ER308	04844 04184	1-211-8	10-10-80	KWK	1-8-81
0	WJ	M80	3/32	ER308	C3064	1-211-8	10-10-80	KWK	1-8-81
0	MRW	414	1/16	ER308	04184	1-211-8	10-10-80	KWK	1-8-81
0	MRW	414	3/32	ER308	C3064	1-211-8	10-10-80	KWK	1-8-81
0	WJ	M80	3/32	ER308	C3064	1-211-8	10-10-80	KWK	1-8-81
0	MRW	414	3/32	ER308	C3064	1-211-8	10-10-80	KWK	1-8-81
0	WJ	M80	1/8	ER308	06393	1-211-8	10-10-80	KWK	1-8-81
0	MRW	414	1/8	ER308	06393	1-211-8	10-10-80	KWK	1-8-81
0	WJ	M80	1/8	ER308	06393	1-260-1	10-13-80	KWK	1-8-81
0	MRW	414	1/8	ER308	06393	1-260-1	10-13-80	KWK	1-8-81
0	WJ	M80	1/8	ER308	06393	1-260-1	10-14-80	KWK	1-8-81
0	JLH	H98	1/8	ER308	06393	1-260-1	10-14-80	KWK	1-8-81
0	WJ	M80	1/8	ER308	06393	1-260-1	10-14-80	KWK	1-8-81
0	JLH	H98	1/8	ER308	06393	1-260-1	10-14-80	KWK	1-8-81
0	WJ	M80	1/8	ER308	06393	1-260-1	10-15-80	KWK	1-8-81
0	JLH	H98	1/8	ER308	06393	1-260-1	10-15-80	KWK	1-8-81
0	RR	P67	1/16	ER308	04194	1-260-1	10-28-80	KWK	1-8-81
0	RR	P67	3/32	ER308	C3064	1-260-1	10-28-80	KWK	1-8-81
0	RR	P67	3/32	ER308	C3064	1-260-1	10-29-80	KWK	1-8-81
0	RR	P67	1/8	ER308	06393	1-260-1	10-29-80	KWK	1-8-81
0	FRH	H06	1/8	ER308	06393	1-260-1	12-10-80	KWK	1-8-81
0	RRB	X89	1/8	ER308	06393	1-260-1	12-10-80	KWK	1-8-81
0	FRH	H06	1/8	ER308	06393	1-260-1	12-11-80	KWK	1-8-81
0	RRB	X89	1/8	ER308	06393	1-260-1	12-11-80	KWK	1-8-81
0	FRH	H06	1/8	ER308	06393	1-260-1	12-12-80	KWK	1-8-81
0	RRB	X89	1/8	ER308	06393	1-260-1	12-12-80	KWK	1-8-81
0	FRH	H06	1/8	ER308	06393	1-304-11	12-12-80	KWK	1-8-81
0	RRB	X89	1/8	ER308	06393	1-304-11	12-12-80	KWK	1-8-81
0	FRH	H06	1/8	ER308	06393	1-304-11	12-12-80	KWK	1-8-81
0	FRH	H06	3/32	ER308	06393	1-304-11	12-12-80	KWK	1-8-81

RECEIVED CHECK SIGNATURE

Ans

3/21/81

25539
31/21/79

FORM M-4A REVISION 1

DUKE POWER COMPANY
CONSTRUCTION DEPARTMENT
PROJECT LATAWBA

M-4A CONTINUATION SHEET

UNIT 2 SYSTEM ALC SUBASSEMBLY/DRAWING NO CN-2NC-13 WELD NO 2NC-13-2

QTY	WELD METAL	WELD SIZES	FILLER SIZE	FILLER TYPE	PLATE/ROOT NUMBER	WELD SIZES	DATE	WELDER	DATE
0	RRB	X89	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-15-80	KWK	1-8-81
0	RRB	X89	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-15-80	KWK	1-8-81
0	FRH	H06	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-15-80	KWK	1-8-81
0	FRH	H06	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-15-80	KWK	1-8-81
0	RRB	X89	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-15-80	KWK	1-8-81
0	RRB	X89	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-15-80	KWK	1-8-81
0	FRH	H06	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-16-80	KWK	1-8-81
0	FRH	H06	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-16-80	KWK	1-8-81
0	RRB	X89	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-16-80	KWK	1-8-81
0	RRB	X89	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-16-80	KWK	1-8-81
0	FRH	H06	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-16-80	KWK	1-8-81
0	FRH	H06	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-16-80	KWK	1-8-81
0	RRB	X89	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-16-80	KWK	1-8-81
0	RRB	X89	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-16-80	KWK	1-8-81
0	RRB	X89	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-17-80	KWK	1-8-81
0	RRB	X89	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-17-80	KWK	1-8-81
0	FRH	H06	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-17-80	KWK	1-8-81
0	FRH	H06	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-17-80	KWK	1-8-81
0	FRH	H06	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-17-80	KWK	1-8-81
0	FRH	H06	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-17-80	KWK	1-8-81
0	RRB	X89	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-17-80	KWK	1-8-81
0	RRB	X89	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-17-80	KWK	1-8-81
0	RRB	X89	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-18-80	KWK	1-8-81
0	RRB	X89	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-18-80	KWK	1-8-81
0	FRH	H06	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-18-80	KWK	1-8-81
0	FRH	H06	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-18-80	KWK	1-8-81
0	FRH	H06	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-18-80	KWK	1-8-81
0	FRH	H06	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-18-80	KWK	1-8-81
0	RRB	X89	$\frac{3}{32}$	E 308-16	8680E	L 304-11	12-18-80	KWK	1-8-81
0	RRB	X89	$\frac{1}{8}$	E 308-16	8665	L 304-11	12-18-80	KWK	1-8-81

TOTAL WELDED JOINTS

0015

3/21/81

Attach A pg 5

CNC-2201.01-00-0006

rw 0

25589
11/21/79

FORM M-4L REVISION 1

LUKE POWER COMPANY
CONSTRUCTION DEPARTMENT
PROJECT CATAUGA

M-4A CONTINUATION SHEET

UNIT 2 SYSTEM NC SUBASSEMBLY/DRAWING NO. CA-2NC-13 WELD NO 2NC13.2

WELD NO	WELD INITIAL	PENCIL NUMBER	FILLER SIZE	FILLER TYPE	WELD LOT NUMBER	FILLER WELD DATE	WELD DATE	WELD DATE
0	FRH	H06	1/8	E 308-16	8665	1-30-11	12-19-80	KWK 1-8-81
2	FRH	H06	3/32	E 308-16	8680E	1-30-11	12-19-80	KWK 1-8-81
0	RRB	X89	3/32	E 308-16	8680E	1-30-11	12-19-80	KWK 1-8-81
0	RRB	X89	1/8	E 308-16	8665	1-30-11	12-19-80	KWK 1-8-81
0	FRH	H06	1/8	E 308-16	8665	1-30-11	12-19-80	KWK 1-8-81
0	FRH	H06	3/32	E 308-16	8680E	1-30-11	12-19-80	KWK 1-8-81
0	RRB	X89	3/32	E 308-16	8680E	1-30-11	12-19-80	KWK 1-8-81
0	RRB	X89	1/8	E 308-16	8665	1-30-11	12-19-80	KWK 1-8-81
0	RRB	X89	1/8	E 308-16	8665	1-30-11	12-22-80	KWK 1-8-81
0	RRB	X89	3/32	E 308-16	8680E	1-30-11	12-22-80	KWK 1-8-81
0	FRH	H06	3/32	E 308-16	8680E	1-30-11	12-22-80	KWK 1-8-81
0	FRH	H06	1/8	E 308-16	8665	1-30-11	12-22-80	KWK 1-8-81
0	FRH	H06	1/8	E 308-16	8665	1-30-11	12-22-80	KWK 1-8-81
0	FRH	H06	3/32	E 308-16	8680E	1-30-11	12-22-80	KWK 1-8-81
0	RRB	X89	3/32	E 308-16	8680E	1-30-11	12-22-80	KWK 1-8-81
0	RRB	X89	1/8	E 308-16	8665	1-30-11	12-22-80	KWK 1-8-81
0	RRB	X89	1/8	E 308-16	8665	1-30-11	12-29-80	KWK 1-8-81
0	RRB	X89	3/32	E 308-16	8680E	1-30-11	12-29-80	KWK 1-8-81
0	FRH	H06	3/32	E 308-16	8680E	1-30-11	12-29-80	KWK 1-8-81
0	FRH	H06	1/8	E 308-16	8665	1-30-11	12-29-80	KWK 1-8-81
0	FRH	H06	3/32	E 308-16	8680E	1-30-11	12-29-80	KWK 1-8-81
0	RRB	X89	3/32	E 308-16	8680E	1-30-11	12-29-80	KWK 1-8-81
0	RRB	X89	1/8	E 308-16	8665	1-30-11	12-29-80	KWK 1-8-81
0	RRB	X89	1/8	E 308-16	8665	1-30-11	12-30-80	KWK 1-8-81
0	RRB	X89	3/32	E 308-16	8680E	1-30-11	12-30-80	KWK 1-8-81
0	FRH	H06	3/32	E 308-16	8680E	1-30-11	12-30-80	KWK 1-8-81
0	FRH	H06	1/8	E 308-16	8665	1-30-11	12-30-80	KWK 1-8-81
0	RRB	X89	1/8	E 308-16	8665	1-30-11	12-30-80	KWK 1-8-81
0	RRB	X89	3/32	E 308-16	8680E	1-30-11	12-30-80	KWK 1-8-81

add

add

Attach A pg 6

WC-2201.01-00-0006

~ 0

*Problem Investigation Process
Catawba Nuclear Station*

PIP Serial No:	Action Category:	LER No:	Other Report:
C-04-05421	3		

Problem Identification

Discovered Time/Date: 10:33 10/11/2004 Occurred Time/Date: 10/07/2004

Unit(s) Affected:

Unit	Mode	%Power	Unit Status	Remarks
2	NOMODE			

System(s) Affected:

NC Reactor Coolant

Affected Equipment

(No Equipment Affected)

Location of Problem:

Bldg: RX Column Line: Elev:

Location Remarks:

S/G 'C'

Method Used to Discover Problem:

RT (ISI) of S/G 'C' Inlet/outlet nozzle weld

Brief Problem Description:

Linear indication of 1 inch found in S/G 'C' on Hot leg during RT(radiography)

Detail Problem Description:

Linear indication of 1 inch found in S/G 'C' hot leg nozzle weld during RT. Linear indication of 1 inch exceeds the acceptance standards of NDE-12, ASME Code Sections III and XI. (Procedure NDE 12 Rev. 11, Appendix A, ASME Code XI IWB-3514, ASME Code Section III NB-5320). Slag indication of 1 inch in length located at RT film interval number '0'. The indication was not visible on the construction radiographs. A change in the source position in relation to the plane of the flaw was determined to be the basis for the flaw detection in 2004 and not during the fabrication RT. This is a reportable indication.

Originated By: JFB8158: BUMGARNER, JAMES F Team: JLW3805 Group: IWS Date: 10/11/2004

Other Units/Components/Systems/Areas Affected(Y,N,U): U

Industry Plants Affected(Y,N,U): U

Immediate Corrective Actions:

Engineering to perform Fracture Mechanics Analysis.

Originated By: JFB8158: BUMGARNER, JAMES F Team: JLW3805 Group: IWS Date: 10/11/2004

Immediate Corrective Action Documents / Work Orders:

	Indiv	Team	Group	Date
Problem Identified By:	JFB8158	JLW3805	IWS	10/11/2004
Problem Entered By:	JFB8158	JLW3805	IWS	10/11/2004

Screening

Action Category: 3 Root Cause performed? No

*Problem Investigation Process
Catawba Nuclear Station*

EP No:

Other Report Nos:

Event Codes:

F8 Testing

Screening Remarks:

Originated By: MLS9465: STANDRIDGE, MICKEY L Team: PAM7334 Group: SRG Date: 10/11/2004

Assignments:

Responsible Groups(s) for Problem Evaluation: MCE Mechanical/Civil Eng
Responsible Group for Present Operability: N/A
Responsible Group for Report Support Info: N/A
Responsible Group for Reportability: N/A
Responsible Group for Overall PIP Approval: IWS INSPEC. & WELD. SERV.

Signature	Type	Indiv	Team	Group	Date
Screened By:		MLS9465	PAM7334	SRG	10/11/2004

Present Operability

Responsible Group: Status:

Sys/Comp Operable? (Y,N,C,E,T):

Required Mode:

Comments:

No Current Signatures For This Section

Reportability

Responsible Group: Status:

Problem Reportable(Y,N,E):

Reportable Per:

Comments:

No Current Signatures For This Section

Investigation Report:

Responsible Group: Act Date:

Investigator: Group:

Due Date:

*Problem Investigation Process
Catawba Nuclear Station*

Date Due to VP or Sta. Mgr:
Date Regulatory or Agency Rpt Due:
Date Investigation Report Approved:

NRC Cause Codes:

Report Support Info:

Responsible Group:

Status:

No Current Signatures For This Section

Problem Evaluation

Event	Cause Code	Cause Description	Primary	Causing Groups
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Problem Evaluation From: Resp. Group: MCE

Status: ReadyForAccept

OEDB Checked: No

OEDB Comments:

Remarks Comments:

Signature	Type	Indiv	Team	Group	Date
Assigned To:				MCE	10/11/2004
Due Date:		11/10/2004			

Corrective Actions

CA Seq. No: 1

Resp Group	Status	Orig Group	Event Code	Prop CAC	Cause Code
RGC	Open	IWS	F8	A2	YYY

Proposed Corrective Action:

Notify the NRC of the Reportable indication.

Originated By: JFB8158: BUMGARNER, JAMES F Team: JLW3805 Group: IWS Date: 10/11/2004

No Current Signatures For This Section

General: Outage:

Mode:

Other Tracking Processes

Type Number Text

Actual Corrective Action:

Priority: N

Actual CAC:

Status:

Due Date:

*Problem Investigation Process
Catawba Nuclear Station*

Signature Type	Indiv	Team	Group	Date
Assigned To:			RGC	10/11/2004

CA Seq. No: 2

Resp Group	Status	Orig Group	Event Code	Prop CAC	Cause Code
RGC	Open	IWS	F8	A2	YYY

Proposed Corrective Action:

Provide Fracture Mechanics Analysis data to the NRC for their review prior to unit re-start.

Originated By: JFB8158: BUMGARNER, JAMES F Team: JLW3805 Group: IWS Date: 10/11/2004
No Current Signatures For This Section

General: Outage: Mode:

Other Tracking Processes

Type Number Text

Actual Corrective Action:

Priority: N Actual CAC: Status: Due Date:

Signature Type	Indiv	Team	Group	Date
Assigned To:			RGC	10/11/2004

Final and Overall PIP Approval

Responsible Group: IWS Status: Screened

Signature Type	Indiv	Team	Group	Date
Assigned To:			IWS	10/11/2004

Any Supplemental Concurrence Signatures Above Do Not Affect PIP Closure.

Closure Document Type Closure Document No

Attachments

Generic Applicability

Responsible Group: Status:
GO PIP No:

Assessment Remarks:

No Current Signatures For This Section

Failure Prevention Investigation

No FPI Records for this PIP.

*Problem Investigation Process
Catawba Nuclear Station*

Remarks

No Remarks for this PIP.

Maintenance Rule

No Maintenance Rule Records for this PIP.

End of the Document for PIP No: C-4-5421
The status of this PIP is: Screened
The duration of this PIP was: 0 days

Indication Evaluation Report

A	STATION Catawba	UNIT 2	ISI PLAN ITEM NUMBER B05.070.005 B05.130.010	ISI PLAN ID NUMBER 2SGC-INLET-SE 2NC13-02
	NDE METHOD Radiography	NDE PROCEDURE USED NDE 12 REV. 11		INDICATION SERIAL NUMBER S/N 1 (located at RT # 0)
	DESCRIPTION OF ITEM CONTAINING INDICATION Steam Gen Inlet Nozzle to Safe End Weld Steam Gen Nozzle Safe End to Pipe weld		DATE INSPECTED 10/07/2004	EVALUATION NEEDED BY DATE 10/11/2004
	COMMENTS: Slag indication 1.00" in length located at RT film interval number "0". The indication was not visible on the construction radiographs. A change in source position in relation to the plane of the flaw was determined to be the basis for the flaw detection in 2004 and not during the fabrication RT.			
ORIGINATED BY: T.L. Tucker			DATE: 10/11/2004	
B	ACCEPTANCE STANDARD USED: Procedure NDE 12 Rev 11 Appendix A, ASME Code Sect. XI IWB-3514, ASME Code Sect. III NB-5320.			
	CALCULATIONS/COMMENTS: Linear indication length of 1.00" exceeds the acceptance standards of NDE 12, ASME Codes Sect. III and Sect. XI.			
	EVALUATION STATUS = Reportable			
Evaluated By (NDE LEVEL III): T.L. Tucker			DATE: 10/11/2004	
C	PIP SERIAL No: C-04-5421			
	ISI COORDINATOR:		DATE:	
D	ADDITIONAL INSPECTIONS <input type="checkbox"/> NOT REQUIRED <input type="checkbox"/> REQUIRED: ISI PLAN ADDENDA S/N: _____			
	SURVEILLANCE INSPECTIONS <input type="checkbox"/> NOT REQUIRED <input type="checkbox"/> REQUIRED: ISI PLAN ADDENDA S/N: _____			
	ISI PLAN MANAGER _____		DATE _____	
E	DISPOSITION <input type="checkbox"/> ACCEPTABLE IN ACCORDANCE WITH IWB-3600 (FMA) <input type="checkbox"/> REPAIRED <input type="checkbox"/> REPLACED			
	COMMENTS:			
DISPOSITIONED BY (ISI Plan Manager) _____			DATE _____	